

Princeton, Kentucky were inadvertently omitted from the certification.

The intent of the Department's certification is to include all workers of the subject firm adversely affected by imports.

The amended notice applicable to TA-W-31,185 is hereby issued as follows:

"All workers of Willwear Hosiery/Shogren Industries, Marion, North Carolina (TA-W-31,182); Chattanooga, Tennessee (TA-W-31,183); Shogren Industries, Concord, North Carolina (TA-W-31,184); Upper Brookville, New York (TA-W-31,185); Nation Hosiery Mills, Inc., Chattanooga, Tennessee (TA-W-31,184A); and Kentucky Lakes Hosiery Mill, Princeton, Kentucky (TA-W-31,185B) who became totally or partially separated from employment on or after May 23, 1994 are eligible to apply for adjustment assistance under Section 223 of the Trade Act of 1974."

Signed at Washington, D.C. this 31st day of August 1995.

Victor J. Trunzo,

Program Manager, Policy and Reemployment Services, Office of Trade Adjustment Assistance.

[FR Doc. 95-22480 Filed 9-8-95; 8:45 am]

BILLING CODE 4510-30-M

[TA-W-31,077]

Sundstrand Corporation, Electric Power Systems Division, Lima, Ohio; Dismissal of Application for Reconsideration

Pursuant to 29 CFR 90.18(C) an application for administrative reconsideration was filed with the Program Manager of the Office of Trade Adjustment Assistance for workers at Sundstrand Corporation, Electric Power Systems Division, Lima, Ohio. The review indicated that the application contained no new substantial information which would bear importantly on the Department's determination. Therefore, dismissal of the application was issued.

TA-W-31,077; Sundstrand Corporation, Electric Power Systems Div., Lima, Ohio (August 31, 1995)

Signed at Washington, DC this 31st day of August, 1995.

Victor J. Trunzo,

Program Manager, Policy & Reemployment Services, Office of Trade Adjustment Assistance.

[FR Doc. 95-22479 Filed 9-8-95; 8:45 am]

BILLING CODE 4310-30-M

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (95-086)]

Notice of Prospective Patent License

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of Prospective Patent License.

SUMMARY: NASA hereby gives notice that Wessex, L.L.C., of Blacksburg, Virginia, has requested a partially exclusive license to practice the invention described and claimed in U.S. Patent No. 5,296,288 entitled "Protective Coating for Ceramic Materials," which issued to the United States of America as represented by the Administrator of the National Aeronautics and Space Administration on March 22, 1994. Written objection to the prospective grant of a license should be sent to Mr. Harry Lupuloff, Senior Patent Attorney, NASA Headquarters.

DATES: Responses to this Notice must be received by November 13, 1995.

FOR FURTHER INFORMATION CONTACT: Mr. Harry Lupuloff, NASA, Code GP, Washington, DC 20546; telephone number (202) 358-2067.

Dated: September 1, 1995.

Edward A. Frankle,
General Counsel.

[FR Doc. 95-22443 Filed 9-8-95; 8:45 am]

BILLING CODE 7510-01-M

NATIONAL SCIENCE FOUNDATION

Advisory Committee for Education and Human Resources Committee of Visitors; Notice of Meeting

In accordance with the Federal Advisory Committee Act (PL 92-463, as amended), the National Science Foundation announces the following Committee of Visitors meeting:

Name: Committee of Visitors (COV) Review of the Graduate and Minority Graduate Fellowship Programs (1119).

Date & Time: September 25, 1995; 8:00 am to 5:00 pm.

Place: NSF Headquarters, 4201 Wilson Boulevard, Arlington, Virginia.

Type of Meeting: Closed.

Contact Person: Dr. Susan Duby, National Science Foundation, 703/306-1694.

Purpose of Meeting: To provide oversight review of the Graduate and Minority Graduate Fellowship Programs.

Agenda: To carry out Committee of Visitors' review, including examination of decisions on applications, reviewer comments, and other privileged materials.

Reason for Closing: These meetings are closed to the public because the Committee

will be reviewing proposal actions that will include privileged intellectual property and personal information that could harm individuals if they were disclosed. If discussions were open to the public, these matters that are exempt under 5 U.S.C. 552 (b)(c)(4) and (6) of the Government in the Sunshine Act would improperly be disclosed.

Dated: September 5, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95-22407 Filed 9-8-95; 8:45 am]

BILLING CODE 7555-01-M

NUCLEAR REGULATORY COMMISSION

Availability of Draft Application; Format and Content Guidance and Review Plan and Acceptance Criteria for Non-Power Reactors

The U.S. Nuclear Regulatory Commission (NRC) is in the process of developing for Non-Power Reactors (NPRs) a "Format and Content for Applications for the Licensing of Non-Power Reactors" (F&C) and a "Standard Review Plan and Acceptance Criteria for Applications for the Licensing of Non-Power Reactors" (SRP). The NRC has made available drafts of Chapters 2, "Site Characteristics," 3, "Design of Structures, Systems, and Components," 4, "Reactor Description," 7, "Instrumentation and Control Systems," 10, "Experimental Facilities and Utilization," 13, "Accident Analyses," and 18, "High-Enriched Uranium to Low-Enriched Uranium Conversions," of the F&C and SRP documents for comment. Other draft chapters will be made available for comment as they are completed.

Copies of these chapters have been placed in the NRC Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC. Single copies of these documents may be requested in writing from Alexander Adams, Jr., Senior Project Manager, U.S. Nuclear Regulatory Commission, MS: O-11-B-20, Washington, DC 20555. Comments on this chapter should be sent by November 28, 1995, to the Director, Non-Power Reactors and Decommissioning Project Directorate at the above address.

Dated at Rockville, Maryland, this 31st day of August 1995.

For the Nuclear Regulatory Commission.
Seymour H. Weiss,
*Director, Non-Power Reactors and
Decommissioning Project Directorate,
Division of Reactor Program Management,
Office of Nuclear Reactor Regulation.*
[FR Doc. 95-22460 Filed 9-8-95; 8:45 am]
BILLING CODE 7590-01-P

[Docket No. 50-293]

**Boston Edison Company, Pilgrim
Nuclear Power Station; Issuance of
Director Decision Under 10 CFR 2.206**

Notice is hereby given that the Director, Office of Nuclear Reactor Regulation, granted in part and denied in part a Petition dated March 10, 1995 (Petition), filed pursuant to 10 CFR 2.206 by Ms. Mary Elizabeth Lampert and 62 other persons (Petitioners).

The Petition requested that during the March 25, 1995, refueling outage and In-Vessel Visual Inspection conducted by the licensee, certain technical concerns be addressed, and that before Pilgrim goes back on-line, appropriate repairs be made or corrective action be taken, and that the U.S. Nuclear Regulatory Commission (NRC or Commission) discuss the status of such repairs or corrective actions with the public in Plymouth, Massachusetts. The Petition also requested that the NRC terminate its policy of issuing Notices of Enforcement Discretion (NOEDs) and asserted that the NRC has not been enforcing its regulations.

On April 19, 1995, the Director informed the Petitioner that the NRC management and staff was meeting with the Boston Edison Company (licensee) on May 11, 1995, and they would hold a meeting to receive public input on the evening of May 11, 1995. The Petitioner's request to discuss the status of repairs or corrective actions was granted by virtue of the public meeting.

The Director of the Office of Nuclear Reactor Regulation has denied the Petitioners' requests to require repairs and corrective actions before permitting the Pilgrim plant to resume operation, and to terminate the use of NOEDs.

The reasons for this decision are explained in the "Director's Decision Under 10 CFR 2.206," (DD-95-19) which is available for public inspection in the Commission's Public Document Room, in the Gelman Building, Lower Level, 2120 L Street, NW., Washington, DC 20555 and at the Local Public Document Room for the Pilgrim facility at Plymouth Public Library, 11 North Street, Plymouth, Massachusetts 02360.

A copy of the Decision will be filed with the Office of the Secretary for the Commission's review in accordance

with 10 CFR 2.206(c). As provided by this regulation, the Director's Decision will constitute the final action of the Commission 25 days after date of issuance of the Decision unless the Commission, on its own motion, institutes review of the Decision within that time period.

Dated at Rockville, Maryland, this 31st day of August 1995.

For the Nuclear Regulatory Commission.
William T. Russell,
*Director, Office of Nuclear Reactor
Regulation.*

**Appendix A to this Document—
Director's Decision Under 10 CFR
2.206: DD-95-19; Boston Edison
Company, License No. DRP-35**

I. Introduction

Ms. Mary Elizabeth Lampert and 62 other individuals (Petitioners) submitted a Petition dated March 10, 1995, pursuant to 10 CFR 2.206 requesting action with regard to the Pilgrim Nuclear Power Station (Pilgrim), operated by the Boston Edison Company (licensee).

The Petition requested that: (1) during the refueling outage and In-Vessel Visual Inspection scheduled for March 25, 1995, by the licensee, certain technical concerns be addressed, and that before Pilgrim goes back on-line, appropriate repairs be made or corrective action be taken; (2) the U.S. Nuclear Regulatory Commission (NRC or Commission) discuss the status of such repairs or corrective actions with the public in Plymouth, Massachusetts; and (3) the NRC terminate its policy of issuing Notices of Enforcement Discretion (NOEDs) and begin enforcing the regulations again.

As the bases for these requests, the Petitioners identified three groups of technical concerns: (1) age-related deterioration of 25 safety related reactor internals; (2) parts and components "known to be a problem at Pilgrim," including the core shroud, water level indicators, quality assurance for fuel pool cooling system during loss-of-coolant accident/loss of offsite power, motor-operated valves, containment integrity, drywell liner corrosion vulnerability, station blackout vulnerability, and Rosemount transmitters; and (3) parts and components "potentially a problem at Pilgrim," including potential fuel rod corrosion and substandard and/or counterfeit parts. The Petitioners contend that allowing the reactor to operate under a NOED cannot pose less risk to the public health and safety than keeping the reactor shut down until NRC regulations are met.

II. Background

By letter dated April 19, 1995, the NRC acknowledged receipt of the Petition and offered a public meeting, which was held in Plymouth, Massachusetts on May 11, 1995. At that meeting, the results of the licensee's inspections conducted during the outage were discussed.

I have completed my evaluation of the Petition. As explained below, Petitioners

have failed to raise any safety concern which would warrant delaying restart of the Pilgrim Nuclear Power Station (which occurred on June 2, 1995), and the Petitioners' request that the NRC terminate the use of NOEDs is denied.

III. Discussion

A. Age-Related Deterioration of Reactor Internals

Many components inside boiling-water reactor (BWR) vessels (i.e., internals) are made of materials such as stainless steel and various alloys that are susceptible to corrosion and cracking. As materials age, they degrade. This degradation can be accelerated by stresses from temperature and pressure changes, irradiation effects on material properties, chemical interactions, and other corrosive environments. As BWRs age, the amount of cracking is expected to increase. Several cases of internals cracking and degradation have been reported to the NRC over the years. In a number of cases, the NRC has concluded that full power operation of the reactor with time-dependent degradation, related to the operating environment, of reactor vessel internals is acceptable as long as the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) safety margins are satisfied and maintained. In the remaining cases, replacement or repairs were performed on the degraded components or internals. The NRC has met with industry every year since 1988 to review the generic safety implications of reactor internals potentially susceptible to age-related cracking. Additionally, a special industry review group, the Boiling Water Reactor Vessels and Internals Project (BWRVIP), was formed to focus on resolution of reactor vessel and internals degradation.

Several industry standards and regulatory requirements and guidelines are in place to address inservice inspections (ISIs) of reactor components. Moreover, the NRC and industry have responded as new issues emerge. For example, issued Generic Letter (GL) 94-03, "Intergranular Stress Corrosion Cracking of Core Shrouds (IGSCC) in Boiling Water Reactors," "in July 1994 requesting Licensees to inspect their shrouds and provide an analysis justifying continued operation until inspections could be completed. General Electric issued Services Information Letter (SIL) No. 588, "Top Guide and Core Plate Cracking," in February 1995 providing specific recommendations for inspections of BWR top guides and core plates. In addition to addressing emerging the BWRVIP is working on a comprehensive plan that will provide detailed guidance on managing cracking in all BWR internals. The plan will address cracking susceptibility, safety consequences, inspection scope and methodology, flaw evaluation, repair strategies, and mitigation of degradation. Several top level executives and technical staff of the Licensee are on the various BWRVIP committees that are developing generic standards for ISI and repairs.

Petitioners request that 25 components be inspected during the 1995 refueling outage (RFO No. 10), and that they be free of any signs of IGSCC or other kind of fatigue.